



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
REGION III  
2443 WARRENVILLE ROAD, SUITE 210  
LISLE, IL 60532-4352

December 1, 2008

Mr. Michael W. Rencheck  
Senior Vice President and  
Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
One Cook Place  
Bridgman, MI 49106

**SUBJECT: D. C. COOK NUCLEAR POWER PLANT, UNITS 1 AND 2  
EVALUATIONS OF CHANGES, TESTS, OR EXPERIMENTS AND  
PERMANENT PLANT MODIFICATIONS BASELINE INSPECTION  
REPORT 05000315/2008008(DRS); 05000316/2008008(DRS)**

Dear Mr. Rencheck:

On October 30, 2008, the U.S. Nuclear Regulatory Commission (NRC) completed the Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications inspection at your D. C. Cook Nuclear Power Plant, Units 1 and 2. The enclosed report documents the inspection results, which were discussed on October 30, 2008, with Mr. Larry Weber and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document

M. Rencheck

-2-

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Sincerely,

/RA/

David E. Hills, Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket Nos. 50-315; 50-316  
License Nos. DPR-58; DPR-74

Enclosure: Inspection Report 05000315/2008008(DRS); 5000316/2008008(DRS)  
w/Attachment: Supplemental Information

cc w/encl: L. Weber, Site Vice President  
J. Gebbie, Plant Manager  
G. White, Michigan Public Service Commission  
Michigan Department of Environmental Quality  
Planning Manager, Emergency Management and Homeland  
Security Division, Michigan State Police Department  
T. Strong, State Liaison Officer

M. Rencheck

-2-

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cc w/encl: L. Weber, Site Vice President  
J. Gebbie, Plant Manager  
G. White, Michigan Public Service Commission  
Michigan Department of Environmental Quality  
Planning Manager, Emergency Management and Homeland  
Security Division, Michigan State Police Department  
T. Strong, State Liaison Officer

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Letter to Mr. Michael W. Rencheck from Mr. David E. Hills dated December , 2008.

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EVALUATIONS OF CHANGES, TESTS, OR EXPERIMENTS AND  
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Enclosure

NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-315; 50-316  
License Nos: DPR-58; DPR-74

Report Nos: 05000315/2008008(DRS); 05000316/2008008(DRS)

Licensee: Indiana Michigan Power Company

Facility: D. C. Cook Nuclear Power Plant, Units 1 and 2

Location: Bridgman, MI

Dates: October 20, 2008, through October 30, 2008

Inspectors: John M. Jacobson, Senior Reactor Inspector (Lead)  
John V. Bozga, Reactor Inspector  
Mohammad Munir, Reactor Inspector

Observer: Michael Jones, Reactor Inspector

Approved by: David E. Hills, Chief  
Engineering Branch 1  
Division of Reactor Safety

Enclosure

## SUMMARY OF FINDINGS

IR 05000315/2008008(DRS); 05000316/2008008(DRS); 10/20/2008-10/30/2008; D. C. Cook Nuclear Power Plant, Units 1 and 2; Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications.

The inspection covered a two-week announced baseline inspection on evaluations of changes, tests, or experiments and permanent plant modifications. The inspection was conducted by three regional based engineering inspectors. Based on the results of this inspection, no findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process" (SDP). Findings for which the SDP does not apply may be Green or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

### **A. NRC-Identified and Self-Revealed Findings**

#### **Cornerstone: Initiating Events**

No findings of significance were identified.

#### **Cornerstone: Mitigating Systems**

No findings of significance were identified.

#### **Cornerstone: Barrier Integrity**

No findings of significance were identified.

### **B. Licensee-Identified Violations**

No findings of significance were identified.

## REPORT DETAILS

### 1. REACTOR SAFETY

#### **Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity**

#### 1R17 Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications (71111.17)

##### .1 Evaluations of Changes, Tests, or Experiments

###### a. Inspection Scope

From October 20, 2008, through October 30, 2008, the inspectors reviewed ten evaluations performed pursuant to 10 CFR 50.59 to determine if the evaluations were adequate and that prior NRC approval was obtained as appropriate. The inspectors also reviewed 14 screenings where licensee personnel had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. The evaluations and screenings were chosen based on risk significance, safety significance, and complexity. The documents reviewed are listed in the Attachment to this report.

The inspectors used, in part Nuclear Energy Institute (NEI) 96-07, "Guidelines for 10 CFR 50.59 Implementation," Revision 1, to determine acceptability of the completed evaluations, and screenings. The NEI document was endorsed by the NRC in Regulatory Guide 1.187, "Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments," dated November 2000. The inspectors also consulted Part 9900 of the NRC Inspection Manual, "10 CFR Guidance for 10 CFR 50.59, Changes, Tests, and Experiments."

This inspection constitutes ten samples of evaluations and 14 samples of changes screened out as defined in Inspection Procedure 71111.17.

###### b. Findings

No findings of significance were identified.

##### .2 Permanent Plant Modifications

###### a. Inspection Scope

From October 20, 2008, through October 30, 2008, the inspectors reviewed ten permanent plant modifications that had been installed in the plant during the last three years. The modifications were chosen based upon risk significance, safety significance, and complexity. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements, and the licensing bases, and to confirm that the changes did not adversely affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The

inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

The inspectors also used applicable industry standards to evaluate acceptability of the modifications. The list of modifications and other documents reviewed by the inspectors is included in the Attachment to this report.

This inspection constitutes ten samples as defined in Inspection Procedure 71111.17.

b. Findings

No findings of significance were identified.

**4. OTHER ACTIVITIES (OA)**

4OA2 Identification and Resolution of Problems (71152)

.1 Routine Review of Condition Reports

a. Inspection Scope

From October 20, 2008, through October 30, 2008, the inspectors reviewed corrective action process documents that identified or were related to 10 CFR 50.59 evaluations and permanent plant modifications. The inspectors reviewed these documents to evaluate the effectiveness of corrective actions related to permanent plant modifications and evaluations for changes, tests, or experiments issues. In addition, corrective action documents written on issues identified during the inspection were reviewed to verify adequate problem identification and incorporation of the problems into the corrective action system. The specific corrective action documents that were sampled and reviewed by the team are listed in the attachment to this report.

b. Findings

No findings of significance were identified.

4OA6 Management Meetings

.1 Exit Meeting Summary

On October 30, 2008, the inspectors presented the inspection results to Mr. Larry Weber and other members of the licensee staff. The licensee acknowledged the issues presented. The inspectors confirmed that none of the potential report input discussed was considered proprietary.

ATTACHMENT: SUPPLEMENTAL INFORMATION

## **SUPPLEMENTAL INFORMATION**

### **KEY POINTS OF CONTACT**

#### Licensee

L. Weber, Sr. Vice President  
J. Nimtz, Licensing Activity Coordinator  
J. Zwolinski, NRA Manager  
M. Scarpello, Licensing Manager  
S. Saad, Structural Supervisor  
S. Bischof, Principal Engineer  
P. VandeVisse, I&C Design Engineer  
A. Petrowsky, Director, Design Engineering  
A. Parihk, Principal Nuclear Specialist  
W. Wah, EDG System Manager  
R. Neuendorf, Configuration Control Manager

#### Nuclear Regulatory Commission

J. Lennartz, Senior Resident Inspector  
P. LaFlamme, Resident Inspector

### **LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED**

#### Opened, Closed, and Discussed

None

## LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety, but rather, that selected sections of portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

### CALCULATIONS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
12-E-S-ELCP-4Kv-002	Bus Transfer Calculation	4
12-E-S-ELCP-600-003	600 V MCC Control Circuit Voltage Drop	3
1-2-O1-02 Calc 1	RTS/ESFAS Allowable Values and Associated Nominal Trip Setpoints for Extended Surveillance Intervals	2
1-E-N-ELCP-4KV-001-VOLT	Unit 1 Voltage Adequacy Analysis	0
PS-4KVP-009	4-kV RCP Bus Under-frequency Relay Settings	0
PS-4KV-009-ADD	4 KV RCP Under-frequency Relay settings	2/7/2007
MD-01-ESW-108-N	ESW System CTS Heat Exchanger DP Assessment	0
MD-12-CTS-007-N	CTS Heat Exchanger UA Determination for Containment Integrity Analysis	3
MD-12-ESW-106-N	Assessment of Increased Lake Water Temperature on Safety Related and Non-Safety Related Systems	4
MD-12-RH-212-N	Determination of Water Inertia Loads for Miscellaneous ECCS Valves	6
MD-02-RH-022-N	Analysis of Thrust and Torque Limits for MOV2-IMO-314	3
MD-12-RHR-109-N	EPRI PPM Evaluation of ½ - IMO-314 & 324	3
MD-02-ECCS-003-N	D.C.Cook, U2 ECCS and CVCS Proto-Flow Model	4
MD-02-ECCS-005-N	U2 ECCS Pump NPSH Analysis	5
DC-D-02-CS-14	Piping and Pipe Support Analysis of the Chemical and Volume Control System from the RCP Cold Leg No. 4 to Regenerative Heat Exchanger and 2" Waste Disposal line to the Penetration Wall CPS-7	2
SD-991014-001	Pipe Supports 2-GSI-V62 and 2-GSI-V61 for 10" SI overflow pipe	1
SD-000429-020	Qualification of pipe supports for 10" SI overflow pipe	1
SD-000502-004	Piping Stress Analysis and support load determination for the 10" overflow line off the	1

## CALCULATIONS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
SD-990913-006	RWST per DCP-1-DCP-678 Piping Stress Analysis and support load determination for the 10" overflow line off the RWST per DCP-2-DCP-678	2
MD-12-RWST-001-N	Maximum Differential Pressure for RWST Vent Path	3
DC-D-01-SI-2	Piping and Pipe Support Analysis of the Safety Injection System (SI)	2
DC-D-02-RC-2306	Pipe Stress Analysis and Support Qualification for the RCS Mid-Loop Level Monitoring System Piping from the Pressurizer Relief Valve Discharge Line to Sight Glass NGG-100 and Instrument NLI-112	4
DC-D-01-RC-5365	Piping and Pipe Support Analysis of the Pressurizer Safety Valve and the Power Operated Relief Valve Discharge Piping System from the Pressurizer to the Pressurizer Relief Tank	1
SD-010321-001	Determine the Structural Adequacy of the Anchorage of the East and West Motor Driven Auxiliary Feedwater Pumps, the Turbine Driven Auxiliary Feedwater Pump, and the enclosure of the West Motor Driven Auxiliary Feedwater Pumps	0
SD-060213-001	Piping and Support Structural Evaluation for Unit 1 Containment Recirculation Sump Vent Line in Support of Modification 1-MOD-65754	1
SD-060307-001	Qualification of Mounting Details for Containment Recirculation Sump Level Switches for 1-MOD-65754	1
DC-D-02-CCW-31	LBPRP-Piping and Pipe Supports Analysis of the CCW Piping System	2
DC-D-01-RH-01	Piping and Pipe Support Analysis of the RHR, SI, CTS and CVCS Piping from the RWST to Pumps 1-PP-050E/W (CCP), 1-PP-009E/W (CTS), 1-PP-049 (Reciprocating Charging), 1-PP-035E/W (RHR), 1-PP-026N/S (SI) and Containment Penetrations 1-CPN-16, 1-CPN-45, 1-CPN-46, 1-CPN-47, 1-CPN-48, 1-CPN-49, 1-CPN-50 and 1-CPN-51 for the Large Bore Piping Reconstitution Program	3

**CORRECTIVE ACTION PROGRAM DOCUMENTS ISSUED DURING INSPECTION**

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
AR 08295053	Admin Errors in Modification Package	10/22/2008
AR 08298014	Insufficient Documentation in Modification Package	10/27/2008
AR 08296062	Admin Errors in 50.59 Screening	10/22/2008
IR 08301024	Typo-Discrepancy in analysis for calculation DC-D-02-CCW-31	10/27/2008
IR 08303072	Discrepancies – pipe support 1-ARH-R4011-Calc DC-D-01-RH-01	10/29/2008
AR 08304007	Evaluate Fouling Factor / Margin on CTS HX	10/30/2008
AR 08304014	Inspector Notes Piping Calculations Were Hard to Follow	10/30/2008
AR 08304028	NRC Observations of HU Performance Issues During Mod/50.59 Inspection	10/30/2008

**CORRECTIVE ACTION PROGRAM DOCUMENTS ISSUED PRIOR TO INSPECTION**

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
AR 00804005	Fuse Blown for Back Up Power to Load Changer for TR101CD	10/13/2006
AR 00819295	Min Volt at Relay Should be Used Not Min Batt Volt in EC	9/21/2007
CR 04175023	Single Point Vulnerability Analysis RCP Trip on Bus Under-frequency	06/23/2004
AR 00840100	Loss of Configuration EDG Fuel Injection Pumps	10/10/08
AR 00826854	Loss of Configuration EDG Fuel Injection Pumps	02/22/2008
ESAT 08284025	Loss of Configuration EDG Fuel Injection Pumps	10/13/2008
ESAT 08270024	Power supply not installed per RTO mod package EC-47709	10/03/2008
IR 00801589	Answer to 1 <sup>st</sup> two 50.59 Eval questions appear incorrect	8/15/06
IR 00823302	Two detail errors were identified in AD 2006-0339-00	12/12/07

**DRAWINGS**

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
OP-2-98050-24	Reserve Bus Tran. And Auxiliary Buses Low Voltage Protection Elem. Diagram	24
OP-2-98041-31	4 KV Aux Transformers 2AB & 201AB Elementary Diagram	31
OP-2-98045-35	4 KV/600 V Auxiliary Transformers 21A & 21C Elementary Diagram Sheet 1 of 2	35
OP-2-98046-29	4 KV 600 V Auxiliary Transformers 21B & 21D Elementary Diagram Sheet 1 of 2	29
OP-2-12001-42	Main Auxiliary One-Line Diagram Bus "A" & "B" ESF Train B	42
OP-2-12002-37	Main Auxiliary One-Line Diagram Bus "C" & "D" ESF Train A	37

## **DRAWINGS**

<b><u>Number</u></b>	<b><u>Description or Title</u></b>	<b><u>Date or Revision</u></b>
OP-2-12003-27	250V DC Main One-Line Diagram ESF Train A, B, N & BOP	27
OP-1-98002-3	Transformer 101 AB Temperature & Cooling Elementary Diagram	3
OP-1-98003-2	Transformer 101 AB Load Tap Changer Motor Drive Control Elem Diagram	2
OP-1-98002-E047420	Transformer 101 AB Temperature & Cooling Elementary Diagram	0
OP-1-98004-2	Transformer 101 AB Load Tap Changer Control Elem Diagram	2

## **50.59 EVALUATIONS**

<b><u>Number</u></b>	<b><u>Description or Title</u></b>	<b><u>Date or Revision</u></b>
2007-0375-00	Loss of All AC Power	1/14/2008
2007-0200-00	Open RHR Cross-tie Modification	0
2008-0177-00	Implementation of New Recirculation Sump Design and Licensing Basis Requirements	0
2007-0142-01	Using VCT Pressure to Adjust #23 RCP No. 2 Seal Leakoff	2
2006-0325-00	Change Tech Spec Basis 3.7.9 to Increase UHS Temperature From 85.8 to 86.8 deg. F	0
2007-0210-01	Unit 2 Containment Sump Remote Strainer Modification	10/31/2007
2006-0224-00	Containment Sump Debris Accumulation Modifications	8/29/2006
2007-0422-00	Unit 1 Containment Sump Remote Strainer Modification	1/10/2008
2007-0104-00	Unit 2 Containment Sump Modification	5/2/2007
2007-0210-00	Unit 2 Containment Sump Remote Strainer Modification	9/10/2007

## **50.59 SCREENINGS**

<b><u>Number</u></b>	<b><u>Description or Title</u></b>	<b><u>Date or Revision</u></b>
2007-0371-00	Time Response and Verification of Engineered Safety Features	10/29/2007
2007-0039-00	Update FSAR Section 8.7.2.4 for CRID Temperatures	2/2/2007
2007-0172-00	Revise RAT LTC Control Power Transfer Schemes	8/29/2008
2007-0070-00	Compensatory Measure in Support of Control Room Habitability and Offsite Dose Analyses	3/2/2007
2007-0567-00	Unit 2 Degraded Voltage Back-Fit Modification	0
2008-0180-00	Modify SDG Auto-start Relay	5/9/2008
2008-0131-00	Change Unit 1 Design Basis ESW Flow to CTS HX	4/7/2008
2007-0339-00	SI Termination 1(2) OHP-4023-ES-1-1 Rev 17/16	10/11/2007
2007-0252-00	Malfunction of a Reactor Coolant Pump	8/10/2007

## 50.59 SCREENINGS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
2008-0025-00	Revision 5 of EHI-5300 is Removing Requirement to LLRT CPN-45 and 46	1/23/2008
2008-0240-00	Change of Setpoint for 2-CFA-454 Low Flow	7/2/2008
2007-0402-00	Change Pipe Classification Downstream of 1-SI-176 and 2-SI-176 from B-14 to B-12 Establishing ISI Class Break at Downstream Side of Valve	12/5/2007
2007-0362-00	Update of ESW Pump Suction Elevation NPSHa and Water Temperature Values in UFSAR Table 6.1-1	10/30/2007
2007-0362-00	Update ESW Pump Suction Elevation NPSHa and Water Temperature Values Listed in UFSAR Table 6.1-1	0

## MODIFICATIONS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
EC 47412	Degraded Voltage Back-Fit Order Modification	0
	SPV Unit 1 Reactor Coolant Pump UF Breaker Trip	0
EC 475	Elimination	
EC 48860	Change U1 Design Basis Flow to CTS Heat Exchanger	0
EC 47742	RHR Open Crosstie Modification for U2	0
EC 48922	Provide New RHR Pipe Support for Partially Removed Support 1-GRH-R-508	0
EC 48343	Add Weld to Pipe Support 2-GCS-R511 and accept as-found member size	0
EC 48138	Permanently flange Unit 2 Refueling Water Storage Tank (RWST) 10 inch overflow line vent	0
EC 48650	Remove portions of the Pipe Whip Restraints located around the Pressurizer PORV manifold piping to provide access for the installation of the PORV inverted Loop Seals per EC-0000047618 and Abandon In Place the remaining Pipe Whip Restraint Members	0
EC 46990	U2 Refueling Water Storage Tank 8" vent pipe modification	0
EC 47058	Install Loop Seals Between the Pressurizer PORVs and the Pressurizer PORV Block Valves	0

## PROCEDURES

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
PMP-5040-MOD-010	Field Change Notices	5
PMP-5040-MOD-007	Engineering Modifications	7

## PROCEDURES

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
2-OHP-4021-054-001	Operation of Condensate System	19
1-IHP-4030-182-003	Reactor Coolant Pump (4 KV) Bus 1B Channel 1 Under-frequency Relay Channel Calibration	2
1-OHP-4023-ECA-0	Loss of All AC Power	24
PMP-2291-PLN-001	Work Control Activity Planning Process	31
12-MHP-5021-032-041	Emergency Diesel Engine Bendix Fuel Injection Pump Removal and Installation	7

## OTHER DOCUMENTS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
WO 55288414-14	Unit 2 Degraded Voltage Back-Fit Order Post Modification Test	10/14/2007
WO 55288414-01	Unit 2 Degraded Voltage Back-Fit Order Modification Bench Test of Relays	9/17/2007
WO 55288414-33	2-62-T21AB, Implement FCN-47412-007 for Train 'B'	10/3/2007
WO 55288414-34	2-62-T21CD, Implement FCN-47412-007 for Train 'A'	10/7/2007
FCN-47412-007	Dropping Resistor as Shown on OP-2-98050 Does Not Provide the Correct Operating Voltage for the NTS 812 Relay	10/2/2007
WO 55294769-01	Replace four Unit 2 AB EDG Fuel Injection Pumps	8/31/2007
WO 55230061-01	2-PS-CGC14A-PS 1/2 Replace Power Supply	9/27/2007
WO 55230061-02	2-PS-CGC14A, Burn in Bench Check Power Supply	5/1/2007
WO 55230061-03	2-PS-CGC14A-PS1/2, Perform "As Left"	10/12/2007
FCN-47412-008	FCN-47412-007 Could Not be Installed as Written	10/3/2007
NTS Manual 812-01	NTS Series 812 Timing Relays	2
VTD-ASEA-0031	ABB Power Distribution Instructions for Single Phase Voltage Relays	2
EC-0000048573 Page 14 of 31	Operating Voltage Agastat 9412	0
2-E-N-ELCP-250-007 Att. AB	250 VDC Battery 2AB System Analysis	2
2-E-N-ELCP-	Path Voltage Drop Summary	01/19/2000 Attachment

## **OTHER DOCUMENTS**

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<b><u>Number</u></b>	<b><u>Description or Title</u></b>	<b><u>Date or Revision</u></b>
250-006 Att. B Page B 20 of 78		
2-E-N-ELCP- 250-007 Att. B	Path Voltage Drop Summary	01/20/2000
ES-ESFTRT- 0001-QCN	ESF Time Response Basis Specification	3/28/2008
WO 55292116- 02	CTS Heat Exchanger 1-HE-18W Shell Side Visual Inspection (including video)	4/4/2008
WO 55292116	Open Shell Side of HTX for Inspection	4/4/2008

## LIST OF ACRONYMS USED

AC	Alternating Current
ADAMS	Agency Wide Document Access and Management System
AISC	American Institute of Steel Construction
ASME	American Society of Mechanical Engineers
CFR	Code of Federal Regulations
CTS	Containment Spray System
CVCS	Chemical and Volume Control System
DRP	Division of Reactor Projects
DRS	Division of Reactor Safety
ECCS	Emergency Core Cooling System
ESW	Essential Service Water
HTX	Heat Exchanger
IMC	Inspection Manual Chapter
IR	Inspection Report
Kv	Kilovolt
MCC	Motor Control Center
NEI	Nuclear Energy Institute
NPSHa	Net Positive Suction Head Available
NRC	U.S. Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
OA	Other Activities
PARS	Publicly Available Records System
RCP	Reactor Coolant Pump
RHR	Residual Heat Removal
ROP	Reactor Oversight Process
RWST	Refueling Water Storage Tank
SDP	Significance Determination Process
SI	Safety Injection
TS	Technical Specifications
UFSAR	Updated Final Safety Analysis Report
UHS	Ultimate Heat Sink
VCT	Volume Control Tank
VDC	Volts Direct Current